

# Contents

## Part I Introductory Systems and Plasma Fundamentals

<b>1</b>	<b>Introduction to Systems Approaches to Nuclear Fusion . . . . .</b>	<b>3</b>
1.1	Fusion Physics and Systems Approaches . . . . .	3
1.1.1	Fusion Reactors and Systems . . . . .	3
1.1.2	Detailed Monograph Organization . . . . .	5
1.1.3	A Simplified Description of Plasmas . . . . .	7
1.1.4	Tokamak Magnetic Field Coils and Geometry . . . . .	9
1.1.5	Plasma Current and MHD Fluid Equilibrium . . . . .	11
1.1.6	Plasma Pressure and Confinement Time . . . . .	12
1.1.7	Individual Particle Effects in Toroidal Systems . . . . .	13
1.1.8	Auxiliary Heating and Current Drive . . . . .	14
1.1.9	Elements of a Tokamak Fusion Reactor . . . . .	14
1.2	Systems Engineering and Architecture Principles . . . . .	16
1.2.1	Elements, Relationships and Systems Thinking . . . . .	16
1.2.2	Hierarchy . . . . .	16
1.2.3	Aspects . . . . .	17
1.2.4	Models . . . . .	17
1.2.5	Agility . . . . .	19
1.3	Systems Engineering Process Model . . . . .	19
1.3.1	Problem Solving Process, Systems Architecture and Design . . . . .	21
1.3.2	Concept Development . . . . .	21
1.3.3	Constraints . . . . .	22
1.3.4	Metrics and Risk . . . . .	22
1.4	Systems Emergent Properties, Robustness and Dynamics . . . . .	23
1.4.1	Emergent Properties of Systems . . . . .	23
1.4.2	Mechanisms for Robustness . . . . .	24
1.4.3	Robustness Trade-Offs . . . . .	24
1.4.4	Fragilities . . . . .	25

1.4.5	Resource Demands . . . . .	25
1.4.6	Performance . . . . .	25
1.5	Examples of Systems Engineering, Architecture, and Emergent Properties . . . . .	26
1.5.1	Building a House . . . . .	26
1.5.2	Landing on the Moon and Returning . . . . .	26
1.5.3	Guiding an Airplane in Flight . . . . .	26
1.6	Systems Reverse Engineering . . . . .	27
1.6.1	Identifying Elements and Relationships in Existing Devices . . . . .	27
1.6.2	Analysing Metrics and Emergent Properties . . . . .	27
1.6.3	Contributing Lessons Learned to the Design Space . . . . .	28
1.7	Systems Forward Engineering – From Concept to Architecture . . . . .	28
1.7.1	Developing a Solution-Neutral Function . . . . .	28
1.7.2	Developing and Implementing Concepts . . . . .	28
1.7.3	Organization of Project Management . . . . .	29
1.8	Summary Checklists of Systems Strategies for Fusion . . . . .	30
1.8.1	Systems Concepts Summary – Checklist 1.1 . . . . .	30
1.8.2	Analyse Existing Plasma Physics and Nuclear Fusion Machines – Checklist 1.2 . . . . .	30
1.8.3	Design, Fund, Build and Commission New Fusion Machines – Checklists 1.3A and 1.3B . . . . .	30
1.8.4	Design and Carry Out Fusion Experiments on Existing Machines – Checklist 1.4 . . . . .	31
1.8.5	Analyse Prototype Reactor Machines and Experiments – Checklist 1.5 . . . . .	32
1.8.6	Planning and Building Next-Step Fusion Reactor Machines – Checklist 1.6 . . . . .	32
1.8.7	Designing and Building a Fusion Reactor – Checklist 1.7 . . . . .	32
1.8.8	Comparing Approaches to Diverse Concept Fusion reactors – Checklist 1.8 . . . . .	33
1.8.9	Detailed Checklists 1.1 to 1.8 . . . . .	33
	References . . . . .	41
2	<b>Systems Design Space for Tokamak Physics and Engineering . . . . .</b>	<b>45</b>
2.1	Developing a Design Space . . . . .	45
2.2	Detailed Views of Tokamak Design Space . . . . .	46
2.2.1	The Time View . . . . .	46
2.2.2	Time Scales and Design Options . . . . .	47
2.2.3	Plasma View . . . . .	49
2.2.4	Spatial View of Form . . . . .	50
2.3	Engineering and Interface Analysis . . . . .	50
2.3.1	Engineering Aspects . . . . .	50
2.3.2	Interface Analysis . . . . .	51

2.4	Operational Scenarios from Tokamaks with Less Than 700 kA Plasma Current . . . . .	53
2.4.1	Strategy for Developing a Catalogue of Operational Scenarios . . . . .	53
2.4.2	TOSCA . . . . .	55
2.4.3	TCABR, Formerly TCA . . . . .	56
2.4.4	Neutral Beam Heating with PLT and COMPASS . . . . .	57
2.4.5	Tokamak T-10, Successor to T-3 and T-4 . . . . .	58
2.4.6	HL-2A, Formerly ASDEX, and HL-2 M . . . . .	59
2.4.7	START . . . . .	61
2.4.8	GLOBUS-M and GLOBUS –M2 . . . . .	62
2.4.9	QUEST . . . . .	63
2.4.10	General Fusion’s Spector and SLiC . . . . .	64
2.5	Design Space Catalogue of Tokamak Operational Scenarios . . . . .	65
2.5.1	Vacuum Vessel and Component Conditioning . . . . .	65
2.5.2	Plasma Start-Up and Current Ramp . . . . .	65
2.5.3	MHD Equilibrium and High Beta . . . . .	66
2.5.4	Particle and Energy Confinement and H-Modes . . . . .	66
2.5.5	Heating, Current Drive and Their Role in Controlling Instabilities and Disruptions . . . . .	68
2.5.6	Divertor Radiation and Plasma-Wall Interaction . . . . .	69
2.5.7	Plasma Diagnostics and Machine Control . . . . .	70
2.5.8	High Power Tritium Burning and Alpha Particle Heating . . . . .	70
2.5.9	Long Pulse and Steady-State Operation . . . . .	71
2.6	Tokamak Simulation Codes . . . . .	72
2.6.1	Plasma Simulation and Design Codes . . . . .	72
2.6.2	Integrated Workflows of Simulation Codes . . . . .	73
2.6.3	Systems Engineering Codes . . . . .	74
2.7	Component Design Options . . . . .	74
2.8	Integration of the Systems Design Space . . . . .	78
	References . . . . .	78

## Part II High-Current Tokamaks

3	<b>Doublet III/DIII-D and 1–2 MA Tokamaks: Robustness and Adaptation . . . . .</b>	<b>89</b>
3.1	Systems Analysis Strategy for Doublet III/DIII-D . . . . .	89
3.1.1	Overall Strategy . . . . .	89
3.1.2	Application of Checklist 1.2 . . . . .	89
3.1.3	Doublet III Form and Function Choices . . . . .	90
3.2	Doublet III Experiments and Scenarios . . . . .	94
3.2.1	MHD Equilibrium . . . . .	94

3.2.2	Plasma Energy Confinement in Different MHD Equilibria . . . . .	94
3.2.3	Doublet III Scenarios . . . . .	95
3.3	DIII-D Design and Results . . . . .	98
3.3.1	Upgrade of Doublet III to DIII-D . . . . .	98
3.3.2	DIII-D Scenarios . . . . .	100
3.4	DIII-D Systems Analysis of Emergent Properties and Tradeoffs . . . . .	104
3.4.1	Analysis of Emergent Properties . . . . .	104
3.4.2	Systems Control . . . . .	104
3.4.3	Fault-Tolerance . . . . .	104
3.4.4	Modularity . . . . .	105
3.4.5	Decoupling . . . . .	105
3.4.6	Resistance . . . . .	105
3.4.7	Avoidance . . . . .	105
3.4.8	Resource Demands . . . . .	106
3.4.9	Robustness . . . . .	106
3.4.10	Fragility . . . . .	106
3.4.11	Performance and Metrics . . . . .	107
3.5	Asdex-Upgrade . . . . .	107
3.5.1	Asdex-Upgrade Design . . . . .	107
3.5.2	Asdex-Upgrade Scenarios . . . . .	109
3.5.3	Robustness and Performance Analysis . . . . .	110
3.6	Alcator C and C-Mod . . . . .	111
3.6.1	Alcator and Alcator C-Mod Design . . . . .	111
3.6.2	Alcator C and C-Mod Scenarios . . . . .	112
3.6.3	Robustness and Performance Analysis . . . . .	115
3.7	FTU . . . . .	116
3.7.1	FTU Design . . . . .	116
3.7.2	FTU Scenarios . . . . .	116
3.7.3	Robustness and Performance Analysis . . . . .	117
3.8	Spherical Tokamaks MAST, MAST-Upgrade and STEP . . . . .	117
3.8.1	MAST Design and Results . . . . .	117
3.8.2	Energy Confinement in MAST . . . . .	118
3.8.3	Robustness and Performance Analysis . . . . .	119
3.8.4	MAST Upgrade First Results . . . . .	119
3.8.5	STEP . . . . .	119
3.9	Spherical Tokamaks NSTX and NSTX-U . . . . .	119
3.10	Low Aspect Ratio with High Vertical Elongation ST-40 . . . . .	120
3.11	Discussion of the Robustness and Performance of 1–2 MA Tokamaks . . . . .	120
	References . . . . .	121

<b>4</b>	<b>TCV: A Case Study in Systems Forward Engineering of a MA Tokamak . . . . .</b>	<b>125</b>
4.1	Systems Forward Engineering Strategy for TCV . . . . .	125
4.1.1	Case Study Strategy . . . . .	125
4.1.2	TCV Capabilities . . . . .	126
4.2	Systems Approaches to the Design of TCV . . . . .	128
4.2.1	Suitability of TCV as a Case Study in Systems Approaches . . . . .	128
4.2.2	Systems Forward Engineering – Getting Started . . . . .	128
4.2.3	Predecessor Tokamak TCA Design . . . . .	129
4.2.4	TCA Technical Choices . . . . .	129
4.2.5	TCV First Goals and Design Options . . . . .	130
4.3	Review and Prioritize Goals and Evaluate Variants for TCV . . . . .	132
4.3.1	Systems Review Process in Early Stages and Prioritizing Goals . . . . .	132
4.3.2	Scenarios for TCV from the Tokamak Design Space and Revised Priorities . . . . .	132
4.3.3	The Intermediate Design Research Areas for TCV . . . . .	133
4.3.4	Tokamak Systems Basic Parameters for Further Design . . . . .	133
4.3.5	TCV Systems Architecture . . . . .	134
4.3.6	Scoping the Systems for Creating High Current and Highly Elongated Plasmas . . . . .	136
4.3.7	Power supply Requirements for Shaping and Controlling Plasma . . . . .	138
4.3.8	Phase I Proposal and Euratom Review . . . . .	140
4.3.9	Phase II Proposal to Euratom . . . . .	141
4.4	TCV Top Level Robustness Properties and Tradeoffs . . . . .	142
4.4.1	Robustness . . . . .	142
4.4.2	Fragility . . . . .	143
4.5	TCV Final Design for Construction . . . . .	144
4.5.1	Phase II Committee Review and Approval . . . . .	144
4.5.2	Systems Level Considerations for Final Design and Construction . . . . .	145
4.5.3	TCV System Level Final Design for Construction . . . . .	145
4.5.4	Implementation . . . . .	146
4.6	TCV Subsystem-Elements Descriptions and Robustness Analysis . . . . .	146
4.6.1	Toroidal Field Coils, Power Supplies and Systems Analysis . . . . .	146
4.6.2	Poloidal Field Coils, Power supplies and Systems Analysis . . . . .	148
4.6.3	Flywheel Motor Generator and Systems Analysis . . . . .	151

4.6.4	Vacuum Vessel and Vacuum Equipment Systems Analysis . . . . .	151
4.6.5	Management of Procurement, Construction and Initial Testing . . . . .	152
4.6.6	Auxiliary Heating Upgrades for TCV . . . . .	153
4.6.7	Machine and Plasma Control and Data Acquisition . . . . .	153
4.7	TCV Scenarios . . . . .	154
4.7.1	High Elongation Plasmas and Ohmic H-Modes . . . . .	154
4.7.2	Fully Digital Plasma Control . . . . .	154
4.7.3	Fully Non-inductive Steady-State Plasmas with Electron Cyclotron Current Drive . . . . .	155
4.7.4	Limits of Operating Space for High Plasma Elongation . . . . .	155
4.7.5	High Power Electron Cyclotron Heating and Bootstrap Current Drive . . . . .	155
4.7.6	Electron Internal Transport Barriers with EC Current Drive . . . . .	156
4.7.7	H-mode Regimes with Ohmic Heating in H, D and He . . . . .	156
4.7.8	Neutral Beam Current Drive and Heating . . . . .	156
4.7.9	Multi-machine ITER Scenario to Maximize Edge Pedestal Height . . . . .	156
4.7.10	Exhaust Control and Detachment in Snowflake and Super-X Divertors . . . . .	157
4.7.11	Vessel Wall Conditioning with ECRH . . . . .	157
4.7.12	Real Time Plasma Control Systems . . . . .	158
4.7.13	Disruption Avoidance by Real-Time Locked Mode Prevention with ECCD . . . . .	158
4.7.14	Elimination of Disruption Runaway Electron Current with Current Ramp Down . . . . .	158
4.7.15	High- $\beta_N$ Fully Noninductive Scenarios with Combined EC and NBI . . . . .	158
4.7.16	Reduced Heat Flux with Grassy ELM's at High Triangularity . . . . .	159
4.7.17	High-Shared-Flux Doublet Configuration with Separate Lobe ECRH . . . . .	159
4.7.18	Removable Gas Baffles Separating Main and Divertor Chambers . . . . .	159
4.8	Lessons Learned About Systems Forward Engineering . . . . .	160
	References . . . . .	160
<b>5</b>	<b>JET – World's Largest Tokamak and its d-t Fusion Experiments Plus TFTR's . . . . .</b>	<b>163</b>
5.1	Systems Analysis Strategy for JET . . . . .	163
5.2	JET Goals, Machine Parameters and Systems Integration . . . . .	164

5.2.1	JET Design and Systems Architecting of Highest Level Systems . . . . .	164
5.2.2	Elements of Next Level Systems, Form Relationships and Design Choices . . . . .	166
5.3	Operational and Experimental Evidence of Success in Machine Operation . . . . .	168
5.3.1	Initial JET Operation Pre-1989 . . . . .	168
5.3.2	High Performance Plasmas in Preparation for Tritium Experiments . . . . .	169
5.3.3	Analysis of Emergent Properties of Systems and Robustness Against Faults . . . . .	171
5.4	Neutron and Fast Particle Diagnostics . . . . .	173
5.4.1	JET Plasma Diagnostics and the Role of Fusion Product Diagnostics . . . . .	173
5.4.2	Fusion Products and Diagnostics . . . . .	173
5.4.3	Measurements with Neutron and Fast Particle Diagnostics . . . . .	175
5.4.4	Robustness Analysis of Neutron and Fast Particle Diagnostics . . . . .	178
5.5	Preliminary Tritium Experiments (PTE) . . . . .	179
5.5.1	Systems Design of Preliminary Tritium Experiments (PTE) . . . . .	179
5.5.2	Preliminary Deuterium-Tritium Fusion Experiments (PTE) . . . . .	181
5.6	Full Power Deuterium-Tritium Experiments (DTE1) . . . . .	184
5.6.1	JET Optimization in Preparation for DTE1 . . . . .	184
5.6.2	Machine Upgrades . . . . .	184
5.6.3	Experimental Preparation for Q = 1 Energy Breakeven Experiments at High Power . . . . .	185
5.6.4	Systems Approaches to Planning Full Power Deuterium-Tritium Fusion Experiments . . . . .	186
5.6.5	Deuterium-Tritium High Power Nuclear Fusion Operational Scenarios and Results in JET . . . . .	193
5.7	Post-DTE1 Operation in JET – Preparation for DTE2 . . . . .	197
5.7.1	Planning for DTE2 Experiments in JET . . . . .	198
5.7.2	Extrapolated Baseline Scenario . . . . .	199
5.7.3	Extrapolated Hybrid Scenario at High Normalized Beta . . . . .	199
5.7.4	Completed Preparations for d-t Experiments . . . . .	199
5.7.5	Successful DTE2 Operation . . . . .	200
5.8	Fusion Power Experiments in TFTR . . . . .	200
5.8.1	Tokamak Fusion Test Reactor (TFTR) Design . . . . .	200
5.8.2	TFTR Experimental Results . . . . .	201
5.8.3	Robustness and Fragility Analysis . . . . .	201
	References . . . . .	202

<b>6</b>	<b>Superconducting and Long-Pulse Tokamaks for Prototyping Reactor Technology</b>	207
6.1	Systems Analysis Strategy for Prototyping Reactor Subsystems	207
6.1.1	Goals and Systems Architecture Choices for Superconducting and Steady-State Tokamaks	208
6.1.2	Key Elements and Constraints of Steady-State Operation	210
6.2	Design Space for Superconducting and Long-Pulse Tokamaks	210
6.2.1	Goals for Using the Design Space	210
6.2.2	Technical Aspects of Superconducting Magnets	211
6.2.3	Systems Architecture of Choices and Trade-Offs	212
6.3	HT-7 (Formerly T-7)	215
6.3.1	Design	215
6.3.2	Scenarios	216
6.3.3	Robustness, Fragility and Relevance to Steady-State Operation	216
6.4	TRIAM-1M	216
6.4.1	Design	216
6.4.2	Scenario: 5 h Fully Non-inductive Steady-State Operation with LHCD	217
6.4.3	Robustness, Fragility and Relevance to Steady-State Operation	218
6.5	T-15	218
6.5.1	Design	218
6.5.2	Scenario: Ohmic Heating in T-15 and Test of Superconducting Windings	218
6.5.3	Robustness, Fragility and Relevance to Steady-State Operation	218
6.6	Tore-Supra, Later WEST, a Case Study of Prototyping a Superconducting Tokamak Reactor	219
6.6.1	Design	219
6.6.2	Scenarios	221
6.6.3	Robustness, Fragility and Relevance to Steady-State Operation	222
6.7	WEST (Tungsten {Symbol “W”} Environment in Steady-State Tokamak)	222
6.7.1	Design	222
6.7.2	Scenario: Plasma and Impurity Confinement in a Long Pulse, High Power, Tungsten Divertor Environment	222
6.7.3	Robustness, Fragility and Relevance to Steady-State Operation	224

6.8	SST-1 (Steady-State Superconducting Tokamak)	224
6.8.1	Design	224
6.8.2	Scenario: LHCD	225
6.8.3	Robustness, Fragility and Relevance to Steady-State Operation	225
6.9	EAST (Experimental Advanced Superconducting Tokamak)	225
6.9.1	Design	225
6.9.2	Scenarios	227
6.9.3	Robustness, Fragility and Relevance to Steady-State Operation	228
6.10	KSTAR (Korea Superconducting Tokamak Advanced Research)	228
6.10.1	Design	228
6.10.2	Scenarios	229
6.10.3	Robustness, Fragility and Relevance to Steady-State Operation	230
6.11	JT-60-U (Japan Tokamak 60 m <sup>3</sup> Plasma: Upgraded JT-60)	230
6.11.1	JT-60-U Design and Relevance to JT-60SA	230
6.11.2	Scenarios	230
6.11.3	Robustness, Fragility and Relevance to Steady-State Operation	232
6.12	JT-60SA (JT-60 Super Advanced)	232
6.12.1	Design	233
6.12.2	Commissioning Until Superconducting Coil Connector Fault	234
6.12.3	Robustness, Fragility and Relevance to Steady-State Operation	234
6.13	Summary Evaluation of Robustness and Fragility in Prototyping Fusion Reactor Subsystems	235
6.13.1	Plasma Start-Up and Current Drive	235
6.13.2	Resistance to Disruptions	235
6.13.3	True Steady-State Operation	235
6.13.4	Long Term Gas Inventory	235
6.13.5	ITER	236
	References	236

### Part III Prototype Tokamak Fusion Reactors

<b>7</b>	<b>ITER: A Fusion Proto-Reactor and its Large Scale Systems Integration</b>	241
7.1	Systems Analysis Strategy for the ITER Tokamak	241
7.1.1	Overall Systems Analysis Strategy	241
7.1.2	ITER Goals	241
7.1.3	ITER Top Level Parameters and Scoping Comparisons	242

7.1.4	Systems Engineering Approaches Used in ITER Design and Construction . . . . .	244
7.2	Review of Top Level Systems Architecture and Innovations . . . . .	246
7.2.1	Overall Systems Architecture . . . . .	246
7.2.2	Innovation . . . . .	247
7.3	Plasma with Heating and Current Drive . . . . .	248
7.3.1	Design Choices . . . . .	248
7.3.2	Robustness, Fragilities and Opportunities . . . . .	251
7.4	Detailed Subsystem Design and Emergent Properties . . . . .	252
7.4.1	Instability Coils . . . . .	252
7.4.2	In-Vessel Diagnostics . . . . .	254
7.4.3	Plasma Facing Components and Neutron Shield . . . . .	254
7.4.4	Test Tritium Breeding Modules (TBM) . . . . .	255
7.4.5	Divertor . . . . .	256
7.4.6	Vacuum Vessel and Disruption Mitigation . . . . .	258
7.4.7	Superconducting Magnetic Coils . . . . .	259
7.4.8	Auxiliary Heating and Current Drive . . . . .	260
7.4.9	Diagnostics and Control for Plasma and Neutrons . . . . .	261
7.4.10	Balance of Plant Including Power Systems, Cooling, Refrigeration and Tritium Processing . . . . .	264
7.5	Systems Aspects of Construction and Operation . . . . .	264
7.5.1	Systems Aspects of Machine Assembly and Commissioning . . . . .	264
7.5.2	Systems Integration of Construction and Operation . . . . .	265
7.5.3	Preparation for Assembly and Commissioning . . . . .	265
7.5.4	Robustness and Fragilities of the Construction Process . . . . .	266
7.5.5	Research Plans to Fulfil Goals . . . . .	266
7.5.6	Robustness and Fragilities of Physics Research Programme . . . . .	267
7.6	Conclusions and Lessons for Future Machines . . . . .	267
7.6.1	Conclusions on Systems Approaches . . . . .	267
7.6.2	Lessons for Future Machines . . . . .	267
	References . . . . .	268
<b>8</b>	<b>Demonstration Tokamak Fusion Reactors and Their Systems Approaches . . . . .</b>	<b>273</b>
8.1	Systems Strategy for Demonstration Tokamak Reactors . . . . .	273
8.1.1	Identify Goals and Essential Elements of a Fusion Reactor . . . . .	274
8.1.2	Paths to an “ITER-based” Reactor . . . . .	275
8.1.3	Key Choices for Neutronics and Reactor Parameters . . . . .	276
8.1.4	Use Systems Codes to Develop Design Analysis Methods . . . . .	279

8.2	The EU DEMO and Planning Major Steps for Designing a Fusion Reactor – Case Study . . . . .	282
8.2.1	Use of Systems Approaches in the Staged Design Approach in Europe . . . . .	282
8.2.2	Application of Systems Codes . . . . .	284
8.2.3	Systems Design Point Studies, Sensitivities and Trade-Offs . . . . .	288
8.2.4	Systems Integration and Key Design Issues . . . . .	289
8.2.5	Fusion Blanket and Shield Thickness for DEMO . . . . .	289
8.2.6	Systems Management Structures . . . . .	292
8.2.7	Robustness and Fragilities . . . . .	292
8.3	Japan Demo and Fusion Reactor Designs and Their Variations . . . . .	293
8.3.1	Current Japan DEMO Design . . . . .	293
8.3.2	Robustness of Japan DEMO Current Design . . . . .	295
8.3.3	Japan DEMO Designs and Options in Systems Architecture . . . . .	295
8.3.4	Summary of Previous Design Variants . . . . .	297
8.4	China Fusion Engineering Test Reactor (CFETR) . . . . .	297
8.4.1	Two Phase Design . . . . .	297
8.4.2	Tritium Fuel Cycle Studies for CFETR . . . . .	297
8.4.3	Toroidal Field Coils . . . . .	298
8.5	Korea K-DEMO . . . . .	298
8.5.1	A High Field Reactor . . . . .	298
8.5.2	Fragility – Cyclotron/Synchrotron Radiation . . . . .	299
8.6	USA DEMO and Fusion Reactors . . . . .	299
8.6.1	A High Elongation Fusion Reactor . . . . .	299
8.6.2	Water or Liquid Nitrogen Cooled Toroidal Field Coil Reactors . . . . .	300
8.6.3	Advanced Tokamak Reactors . . . . .	300
8.7	Large Scale DEMOs – Robustness and Fragilities . . . . .	301
8.7.1	Common Features of Large Scale DEMOs . . . . .	301
8.7.2	Robustness Summary for Large DEMO Machines . . . . .	302
8.8	Very High Magnetic Field Reactors . . . . .	303
8.8.1	SPARC (Soonest/Smallest Private-Funded Affordable Robust Compact) . . . . .	303
8.8.2	ARC (Affordable Robust Compact) . . . . .	305
8.8.3	Innovations Required for Very High Field Reactor . . . . .	307
8.8.4	Fragilities of Very High Field Tokamaks . . . . .	308
8.9	Compact Spherical Tokamak Fusion Reactors . . . . .	310
8.9.1	ST-135 and STEP (Spherical Tokamak for Energy Production) . . . . .	310
8.9.2	USA Sustained High-Power Density (SHPD) Tokamak Facility . . . . .	312

8.9.3	Robustness of Spherical Tokamak Reactors . . . . .	312
8.9.4	Fragilities of Spherical Tokamak Reactors . . . . .	313
8.9.5	A Pulsed Reactor Concept Using Coaxial Helicity Injection . . . . .	313
8.10	Summary . . . . .	314
	References . . . . .	315

## Part IV Helical, Linear and Inertial Fusion Reactor Concepts

<b>9</b>	<b>Helical Fusion Reactor Concepts . . . . .</b>	<b>321</b>
9.1	Introduction to the Stellarator and Heliotron Families . . . . .	321
9.1.1	Systems Analysis Strategy . . . . .	321
9.1.2	Magnetic and Coil Configuration . . . . .	322
9.1.3	Early History of Stellarator Development . . . . .	323
9.1.4	Required Properties of Stellarator Fields . . . . .	324
9.1.5	Main Stellarator Configurations and Design Space Options . . . . .	324
9.1.6	MHD Equilibrium and Stability . . . . .	325
9.1.7	Particle and Energy Transport . . . . .	325
9.1.8	Symmetry and Stellarator Design Space Optimization Criteria . . . . .	327
9.1.9	Coil Errors and Tolerances . . . . .	328
9.2	Wendelstein7-AS (W7-AS) . . . . .	329
9.2.1	W7-AS Systems Architecture and Technical Choices . . . . .	329
9.2.2	W7-AS Stellarator Field Optimization . . . . .	331
9.2.3	W7-AS Operational Scenarios . . . . .	332
9.2.4	W7-AS Goals Achieved, Robustness and Fragilities . . . . .	334
9.3	Wendelstein 7-X: A Case Study in Systems Forward Engineering of a Superconducting Helias . . . . .	334
9.3.1	Systems Analysis Strategy for the Superconducting W7-X Helias as a Case Study . . . . .	334
9.3.2	Systems Approaches to the Design and First Goals of W7-X . . . . .	335
9.3.3	Review and Prioritize Goals and Evaluate Variants for W7X . . . . .	337
9.3.4	W7-X Revised Proposal, Robustness and Tradeoffs . . . . .	339
9.3.5	W7-X Final Design for Construction . . . . .	340
9.3.6	Procurement, Construction, and Initial Testing of W7-X . . . . .	340
9.3.7	W7-X Operational Scenarios . . . . .	341
9.3.8	W7-X Metrics and Performance . . . . .	343

9.4	Heliotron-E . . . . .	343
9.4.1	Heliotron-E Systems Architecture and Technical Choices . . . . .	343
9.4.2	Heliotron-E Operational Scenarios . . . . .	344
9.5	Heliotron-J, a Helical Axis Configuration . . . . .	344
9.5.1	Heliotron-J Systems Architecture and Technical Choices . . . . .	344
9.5.2	Heliotron-J Operational Scenarios . . . . .	345
9.6	Superconducting Large Helical Device (LHD) . . . . .	345
9.6.1	LHD Goals, Systems Architecture and Technical Choices . . . . .	346
9.6.2	LHD Operation Scenarios . . . . .	348
9.6.3	LHD Robustness and Fragilities . . . . .	349
9.7	Alternative Helical Configuration Systems Architecture and Operational Scenarios . . . . .	350
9.7.1	TJ-II Heliac . . . . .	350
9.7.2	Model Validation in Stellarators . . . . .	352
9.7.3	Uragan-2 M and Uragan 3-M Torsatrons . . . . .	352
9.7.4	H-1NF Heliac . . . . .	352
9.7.5	HSX . . . . .	353
9.7.6	WEGA/HIDRA . . . . .	353
9.7.7	Scyllac . . . . .	353
9.7.8	CTH . . . . .	354
9.7.9	NCSX . . . . .	354
9.7.10	Stellarator Robustness and Fragilities of Alternative Architectures . . . . .	355
9.7.11	Comparison of Stellarators and Tokamaks . . . . .	355
9.8	Helias Fusion Reactor Concept HELIAS 5-B . . . . .	356
9.8.1	Basic Design Considerations . . . . .	356
9.8.2	Updated Reactor Design . . . . .	358
9.8.3	A Systems Approach to Optimization . . . . .	359
9.8.4	Detailed Breeding Blanket Design . . . . .	359
9.8.5	Balance of Plant . . . . .	360
9.8.6	Robustness and Fragilities . . . . .	360
9.9	Heliotron Fusion Reactor Concept FFHR-d1 . . . . .	360
9.9.1	Basic Design . . . . .	360
9.9.2	A Systems Approach to Reactor Design and Optimization . . . . .	361
9.9.3	Use of Joints and High Temperature Superconductors . . . . .	362
9.9.4	Helical Divertor . . . . .	362
9.9.5	A Liquid salt Breeding Blanket . . . . .	363
9.9.6	Robustness and Fragilities . . . . .	363
9.9.7	A Helical Volumetric Neutron Source FFHR-b2 . . . . .	364
9.10	Prospects for Stellarators . . . . .	364
	References . . . . .	364

<b>10 Linear Magnetic Traps, Field Reversal and Taylor-State Configurations</b>	371
10.1 Linear Mirror Systems	371
10.1.1 Basic Magnetic Mirror Machine	371
10.1.2 Minimum-B Magnetic Wells	372
10.1.3 Direct Conversion and Reactor Systems Analysis	373
10.1.4 Tandem Mirror TMX and TMX-Upgrade	374
10.1.5 Tandem Mirror PHAEDRUS-B	375
10.1.6 Tandem Mirror GAMMA 10	375
10.1.7 Tandem Mirror KMAX	376
10.2 Mirror Fusion Test Facility (MFTF) and MFTF-B – A Case Study of Systems Forward Engineering	377
10.2.1 Systems Forward Engineering and Architecture in MFTF	377
10.2.2 First Design Improvement of MFTF, One Year Later	378
10.2.3 Systems Review Process in Early Stages	379
10.2.4 Alteration of MFTF to Tandem Mirror MFTF-B	379
10.2.5 Revised Goals of MFTF-B	383
10.2.6 Final Construction Status Report and Project Cancellation	383
10.2.7 Robustness, fragilities and Implications of Cancellation	383
10.3 Gas Traps, Multiple Mirrors, Cusps and Field Reversal	384
10.3.1 Gas Dynamic Trap (GDT)	384
10.3.2 Multiple Mirror GOL-3 and GOL-NB	385
10.3.3 Cusps	386
10.3.4 Tri Alpha Energy C-2U and C-2W Field Reversed Configuration	386
10.4 Linear Fusion Reactors and Systems Design Space	387
10.4.1 Physics of the Systems Architecture Design Space	387
10.4.2 Design-Space Building-Blocks for Systems Architecture of Linear Machines	388
10.4.3 Linear Fusion Reactors	390
10.4.4 Synthesis of Linear Mirror Design Space	391
10.5 Toroidal Versions of Linear Mirror Concepts	392
10.5.1 ELMO Bumpy Torus (EBT)	392
10.5.2 Nagoya Bumpy Torus (NBT-1 M)	392
10.5.3 Auto-injection Mirror	392
10.6 Taylor State $q < 1$ Torus: Spheromak and Reversed Field Pinch	393
10.6.1 Spheromak, Reversed Field Pinch and Taylor State	393
10.6.2 SSPX Spheromak	394

10.6.3 Reversed Field Experiment RFX-Mod and RFX-Mod2	395
10.6.4 MST and Reactor Studies	395
10.6.5 Review and Prospects for RFP	396
References	397
<b>11 Inertial Fusion and Magnetic Fast Pulsed Systems</b>	401
11.1 Systems Analysis Strategy for Laser Inertial Confinement	401
11.1.1 Introduction to Inertial Confinement	401
11.1.2 Progress in Inertial Fusion Driver Concepts	402
11.1.3 Challenges for Inertial Fusion Reactor Development	404
11.1.4 Systems Analysis Strategy	404
11.2 Case Study of Architecture and Forward Engineering of NIF	405
11.2.1 NIF Background	405
11.2.2 NIF Systems Approach	405
11.2.3 NIF Beamline Systems Architecture and Design Space	407
11.2.4 Highly Modular and Robust Construction	409
11.2.5 Operation and Control Systems	409
11.2.6 NIF Physics Results	409
11.2.7 Computer Simulations of Inertial Confinement Laser and Target Interactions	411
11.2.8 Near Breakeven Production of 1.3 MJ of Fusion Energy	411
11.2.9 Laser Inertial Fusion Energy (LIFE) Reactor	412
11.2.10 Overall Fusion Efficiency	412
11.2.11 Laser Mega-Joule (LMJ) and ShenGuang III (SG-III) Large Scale Indirect Drive Lasers	413
11.3 Direct Drive, Fast and Shock Heating and Alternate Laser Technologies	413
11.3.1 PETAL	413
11.3.2 Direct Drive Laser Fusion with OMEGA	414
11.3.3 Gekko and LFEX Lasers for Fast Ignition Realization Experiment (FIREX)	416
11.3.4 The UK Central Laser Facility (CLF) and DiPOLE	418
11.3.5 Petawatt and Exawatt Lasers	418
11.3.6 Non-thermal $p\text{-}^{11}\text{B}$ Fusion from Picosecond Lasers	419
11.4 Design Space and Systems Approaches for Laser Fusion	419
11.4.1 Design Space for a Laser Inertial Confinement Fusion Reactor	419
11.4.2 HiPER Laser Inertial Confinement Fusion Reactor Project	423
11.4.3 Detailed Systems Modelling of HiPER	425
11.4.4 The Future for Laser Fusion Reactors	426



11.5	Magnetic Fast Pulsed Systems . . . . .	427
11.5.1	Dense Plasma Focus . . . . .	427
11.5.2	Plasma-Jet-Driven Magneto-Inertial Fusion (PJMIF) . . . . .	427
11.5.3	MagLIF Z-pinch Experiments . . . . .	427
11.5.4	Z-pinch Fusion Reactor . . . . .	429
11.6	Prospects and Systems Robustness of Inertial Fusion Reactors . . . . .	429
	References . . . . .	430

## Part V Synthesis and Conclusions

### 12 Synthesis and Conclusions on the Applications of Systems

	<b>Approaches to Fusion Reactors</b> . . . . .	435
12.1	Methods Developed and Lessons Learned . . . . .	435
12.2	Overall Systems Analysis Strategy for Comparing Concepts by Systems Architecture . . . . .	438
12.2.1	Methods . . . . .	438
12.2.2	Systems Strategy for Comparing Different Fusion Reactor Concepts . . . . .	439
12.3	Progress in Existing and Planned Machines . . . . .	439
12.3.1	Systems Level Performance Metrics: Lawson Criterion and Triple Product . . . . .	439
12.3.2	The Cost Metric of Fusion Experiments and Reactors . . . . .	440
12.3.3	Metrics, Scaling Laws, Systems and Predictive Codes, Innovation and Breakthroughs . . . . .	442
12.3.4	The Central Role of ITER . . . . .	443
12.4	Pathways Towards an Operating Fusion Reactor . . . . .	444
12.4.1	Pathways Towards a Working Fusion Reactor for Each Concept . . . . .	444
12.4.2	Robustness and Fragilities of Different Concepts . . . . .	445
12.4.3	Tokamak Reactors . . . . .	445
12.4.4	Stellarator, Helias and Heliotron . . . . .	447
12.4.5	Mirrors, Linear Traps and Field Reversal . . . . .	447
12.4.6	Inertial Fusion . . . . .	449
12.5	Overall Conclusions: Optimization of Fusion Reactors by Systems Approaches . . . . .	450
	References . . . . .	450

<b>Glossary</b> . . . . .	453
---------------------------	-----

<b>Index</b> . . . . .	459
------------------------	-----