

INVITED LECTURES

Monday morning

Session A :

Opening session

9.30 h	A I.01	Welcome Address and Opening Remarks	<i>F. Prevot</i>
10.00 h	A I.02	The Evolution of the European Fusion Technology Program Towards NET	<i>H.H. Hennies</i>
11.00 h	A I.03	Status of Tokamak Physics	<i>F. Engelmann</i>
11.45 h	A I.04	The Impact of the First JET Results on the Development of Fusion	<i>P.H. Rebut</i>

Monday afternoon

Session B :

First wall, Divertor, Limiter and Vacuum Technology – Experimental Systems (Part I : posters)

14.30 h	B I.01	Progress in Fusion Technology in the US Magnetic Fusion Program	<i>R.J. Dowling</i>
14.50 h	B I.02	TFTR FIRST WALL Design and Operations Experience	<i>D.K. Owens</i>

Tuesday morning

Session C :

Power Supplies – Plasma Equilibrium – High Frequency Heating Systems

8.30 h	C I.01	Technical Aspects of ECRH Systems	<i>A.G.A. Verhoeven</i>
9.00 h	C I.02	ICRH Results and Technological Limitations	<i>F. Wesner</i>
9.30 h	C I.03	Technical Aspects of Heating and Current Drive by LH Waves on Tokamaks	<i>G. Tonon</i>

Tuesday afternoon

Session D :

Neutral Beam Heating Systems – Materials

14.30 h	D I.01	Long Pulse Neutral Injection for ASDEX, ASDEX UPGRADE and WENDELSTEIN VII – AS. Design, Development and Systems Performance	<i>J.F. Feist</i>
14.00 h	D I.02	Digitized High Power Modulation	<i>U. Schwarz</i>

Wednesday morning

Session E : **Experimental systems (Part II : oral)
Blanket Technology – Refueling – Safety**

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|---------|---------------|--|------------------------|
| 8.30 h | E I.01 | The Wendelstein VII – AS
Experiment | <i>G. Grieger</i> |
| 9.00 h | E I.02 | The FTU, Frascati Tokamak
Upgrade | <i>R. Andreani</i> |
| 9.30 h | E I.03 | The Technical Features
and the progress of
ASDEX UPGRADE | <i>W. Köppendorfer</i> |
| 10.30 h | E I.04 | The RFX Experiment | <i>G. Malesani</i> |
| 11.00 h | E I.05 | Status Report on TORE SUPRA | <i>R. Aymar</i> |

Thursday morning

Session G : **Tritium Handling – Data Acquisition
and Control Systems**

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|--------|---------------|---|-----------------|
| 8.30 h | G I.01 | Tritium Production and Processing
in a Tokamak Reactor | <i>D. Léger</i> |
| 9.10 h | G I.02 | Preparation for DT Phase
Operation in JET | <i>J. Dean</i> |

Thursday afternoon

Session H : **Magnet Technology – Reactor Studies –
Remote Handling**

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|---------|---------------|--|-----------------|
| 14.30 h | H I.01 | PHEBUS : A New and Powerful
Laser Facility for
ICF Research | <i>M. André</i> |
| 15.10 h | H I.02 | Rand D in Support of the
1500 MWe Fast Neutron Reactor
in the Light of the Experience
Gained on SUPERPHENIX | <i>N. Lions</i> |

Friday morning

Session I : **Large Experiments and Projects**

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|---------|---------------|---|----------------------|
| 9.00 h | I I.01 | Technical Aspects of the New
JET Development Plan | <i>M. Huguet</i> |
| 9.20 h | I I.02 | Development of the JET
Poloidal Electromagnetic System | <i>E. Bertolini</i> |
| 9.50 h | I I.03 | First Wall Concepts and
Density Control in JET | <i>K. Sonnenberg</i> |
| 10.20 h | I I.04 | Current Profile
Control in JET | <i>J. Jacquinet</i> |
| 11.20 h | I I.05 | Coils System of
the NET Machine | <i>E. Salpietro</i> |
| 11.50 h | I I.06 | Remote Maintenance of
NET in – Vessel Components | <i>M. Chazalon</i> |
| 12.20 h | I I.07 | Concluding Remarks | <i>R. Toschi</i> |

ORAL PRESENTATION

(in parallel with Poster sessions)

Monday afternoon

Session B :		First wall, Divertor, Limiter and Vacuum Technology – Experimental Systems (Part I : posters)	
16.15 h	B O.01	The Environmental Impact of the Future Fusion Reactors	<i>P. Rocco</i>
17.00 h	B O.02	Overview of the JET ICRF Power Plant Operation and Development	<i>R.J. Anderson</i>
17.15 h	B O.03	Startup Performance of the DIII – D Tokamak	<i>R. Callis</i>

Tuesday afternoon

Session D :		Neutral Beam Heating Systems – Materials	
16.15 h	D O.01	Pellet Fueling Development at ORNL	<i>C.A. Foster</i>
17.00 h	D O.02	TEXTOR Neutral Injection System	<i>U. Pfister</i>
17.15 h	D O.03	Plasma Exhaust Purification Experimental Basis for a Process Concept	<i>R.D. Penzhorn</i>

Thursday morning

Session G :		Tritium Handling – Data Acquisition and Control Systems	
10.00 h	G O.01	Compact Ignition Tokamak Design	<i>A. Flanagan</i>
10.15 h	G O.02	Zero dimensional RFP Ignition Studies	<i>R.A. Scardovelli</i>
11.00 h	G O.03	Reactor Cost Driving Items	<i>W.R. Spears</i>
11.15 h	G O.04	Impact of the T.F. Coil Performance on Cost and Compactness of Next Generation Tokamaks	<i>K. Borrass</i>

Thursday afternoon

Session H :		Magnet Technology – Reactor Studies – Remote Handling	
16.00 h	H O.01	Reliability Characteristics of Major NET Components	<i>R. Bünde</i>
16.15 h	H O.02	Strategy of Breeding Blanket Introduction in NET and Testing Requirements	<i>M. Chazalon</i>
17.00 h	H O.03	The TFTR Maintenance Manipulation Design and Operation Characteristics	<i>G. Boehme</i>
17.15 h	H O.04	Pebble Bed Canister : a Ceramic Breeder Blanket with Helium Cooling for NET	<i>M. Dalle Donne</i>

POSTER PRESENTATION

Monday afternoon

Session B : First wall, Divertor, Limiter and Vacuum Technology – Experimental Systems (Part I : posters)

- 15.35 h B P 01 First Wall Design for TORE SUPRA
- B P 02 Simulation Experiment of First Wall Carbon Coating
- B P 03 Study on Cryopump with High Load
- B P 04 Numerical Analysis of the Eddy Currents and Magnetic Forces in the Vacuum Vessel of ASDEX UPGRADE
- B P 05 Development of the Brazed First Wall Concept for NET
- B P 06 COMPASS Vacuum Vessel Assemblies
- B P 07 Metal Seals for Vacuum Vessels at Temperatures up to 350°C
- B P 08 A First Wall Concept With Radiatively Cooled Protection Tiles
- B P 09 Operation and Performance of the Large Scale JET Cryopump System
- B P 10 Parametric Stress Analysis of NET First Wall Designs
- B P 11 Electromagnetic Effects on the NET First Wall Caused by a Plasma Disruption Event
- B P 12 First Wall Design Criteria. Thermal Fatigue and Creep Experiments and Theoretical Modelling
- B P 13 Thermal Structure of the First Wall and High Heat Removal in Cassette Compact Toroid Reactor
- B P 14 Performance Analysis of a Full Graphite Armor for RFX P.G. Sonato
- B P 15 Surface Conditioning Technique in the FTU (Frascati Tokamak Upgrade) Vacuum Chamber
- B P 16 Evacuation of the NET Vacuum Chamber
- B P 17 Titanium Getter Pumps for TORE SUPRA – N.I. Lines
- B P 18 Concept, Calculation and Design of the Insulation Gap in the Vacuum Vessel of ASDEX UPGRADE
- B P 19 Carbon Protection Tiles for JET
- B P 20 CIT Vacuum Chamber Conceptual Design and Analysis
- B P 21 NET Vacuum Vessel and Bulk Shield Design
- B P 22 Experimental Results on Gas Puffing Control on TUMAN – 3 Tokamak
- B P 23 First Wall Concepts with Welded Steel Structures for NET
- B P 24 The JET Belt Limiter
- B P 25 Surface Analyses of Tic Coated Molybdenum Limiter Material Exposed to High Heat Flux Electron Beam
- B P 26 Experiences with the ASDEX Neutralizer Plates and Construction of Water – Cooled Plates for Long – Pulse Heating
- B P 27 New TEXTOR Liner as a Supporting Structure for the Toroidal Pump Limiter ALT – II
- B P 28 Design for a Pumped Limiter in TFTR
- B P 29 JT – 60 Divertor Pumping System and its Initial Operation in Ohmically Heated Divertor Discharge
- B P 30 A New Divertor Plates Design Concept for the Double Null NET Configuration
- B P 31 Engineering Considerations for the TORE SUPRA Pump Limiter

- B P 40** Engineering Aspect of Helical Axis Torsatron
- B P 41** The Advanced Toroidal Facility (ATF)
- B P 42** The HBTX1B Reversed Field Pinch Torus Assembly
- B P 43** MEQALAC : A Low – ion Accelerator with Application to Fusion Devices
- B P 44** Field Errors in the RFX Magnetic Field Configuration
- B P 45** TPE – 1RM15 Reversed Field Pinch Experiment – Design and Construction –
- B P 46** Description of the Heliac TJ – II and its ECRH System
- B P 47** Status of T – 15 Engineering Systems Manufacture and Assembly

POSTER PRESENTATION

Tuesday morning

Session C : **Power Supplies – Plasma Equilibrium – High Frequency Heating Systems**

- 10.05 h C P 01 Positional control of Tokamak Divertor Plasmas
- C P 02 Equilibrium and Stability Studies on the REPUTE – 1 RFP
- C P 03 Equilibrium and Axisymmetric Stability of the Proposed TCV Tokamak
- C P 04 Plasma Position and Shape Control in TORE SUPRA : Magnetic Measurements and Control Loop
- C P 05 Electromagnetic Effects of the Use of Ferromagnetic Materials Within the NET Machine
- C P 06 NET Plasma Engineering and the Poloidal Field Coil Power Supplies
- C P 07 Development and Testing of Components for the ASDEX ICRF Long – Pulse Heating
- C P 08 The ECRH System for the DITE and COMPASS Experiments
- C P 09 New ICRH System for TEXTOR Compatible with ALT – II
- C P 10 High – Power mm – Wave Transmitter at 60GHz
- C P 11 Progress in ICRF Heating Technology and Designs for Future Large Tokamak Heating Systems
- C P 12 Microwave Engineering for ECRH on TFR
- C P 13 Modular Multijunction Launchers for 3.7 GHz – 8 MW – CW – Lower Hybrid Experiments on TORE SUPRA
- C P 14 Technological Problems and Solutions for the FTU LHRH System
- C P 15 A Ridged Waveguide Proposed for ICRH Heating at 120 MHz in TORE SUPRA
- C P 16 A 3.7 GHz, 500 kW CW Klystron for TORE SUPRA
- C P 17 The TORE SUPRA Lower Hybrid Transmitter
- C P 18 Two Years of Experience with a 600 kW ECRH Installation on TFR
- C P 19 The ASDEX ICRF Antennae in their Uncooled and Cooled Versions
- C P 20 Matching Calculation and Performance in ICRH on ASDEX
- C P 21 Design and Manufacture of Water Cooled Electrostatic Screens for JET
- C P 22 Non Evaporable Getter Pumping for JET ICRF Antennae
- C P 24 Limits of Operation of Conventional RF Windows for Electron Cyclotron Wave Launchers in a Reactor
- C P 25 The 70 GHz/1 MW Long – Pulse ECRH System on the Advanced Stellarator W VII – AS
- C P 26 The Design for High Power Tetrodes for ICRH Applications

- C P 40** Protection and Switching System for the RFX Power Supply
- C P 41** Compass Poloidal Field Power Supply System
- C P 42** Operating of the TEXTOR Accel Power Supply on the Neutral Injection Test Stand
- C P 43** The Development of the JET Poloidal Field Power Supplies for Maximising the Flux Swing Capabilities
- C P 44** The High Voltage Grid Interface in the Present and Future JET Operational Requirements
- C P 45** The TORE SUPRA Poloidal Field System Power Supplies
- C P 46** Operation of the JET Magnet Power Supplies. Reliability and Improvements
- C P 47** Sequential Control of TFTR Power Supplies
- C P 48** The FTU Power Supply System : Final Design
- C P 49** JT – 60 Power Supply Operational Experience
- C P 50** DC Thyristor Switching Elements for Fusion Experiments
- C P 51** ASDEX UPGRADE – Power Supply Systems
- C P 52** ASDEX UPGRADE – OH Circuit
- C P 53** Prototype Tests on the Ion Source Power Supplies of TEXTOR Ni – System
- C P 54** Design, Commissioning and Early Operation of the Power Supply and Protection System for the Extraction Grid of the JET Neutral Injectors at 160 kV.
- C P 55** The JET Neutral Injection 160kV Power Transmission Lines, the Associated Snubbers and the SF6 Tower for the Termination of the Transmission Lines, Housing of the Snubbers and voltage Breaks

POSTER PRESENTATION

Tuesday afternoon

Session D : Neutral Beam Heating Systems – Materials

- 15.35 h D P 01 Neon, Helium and Self – Ion Damage in Nickel
- D P 02 Development of Low Activation Al Base Alloys for Fusion
- D P 03 Corrosion of Steels by Lithium Oxide and Lithium Silicate
- D P 04 Corrosion of 316 Steel by Lithium – Lead Alloys under Nitrogen
- D P 05 The Influence of Helium on the High Temperature Fatigue Life of Austenitic Stainless Steel
- D P 07 Corrosion of AISI 316L Steel in Liquid Lithium in the Presence of a Magnetic Field
- D P 08 Effect of Helium and Fission Neutrons on the Tensile Properties of V – 5% Ti
- D P 09 Irradiation Effects on Mechanical Properties of the NET 316L Reference Heat
- D P 10 Physico Chemical Properties of Porous LiAlO₂
- D P 11 Fundamental Behavior of Shape Memory Alloy Coupler for Quick Replacement Handling of Fusion Core Parts in Cassette Compact Toroid Reactor
- D P 12 The Preparation, Characterisation and Properties of Lithium Oxide and Lithium Metazirconate Specimens Irradiated in HFR Petten in the Second and Third Exotic Experiments
- D P 13 The Corrosion of Type 316 s.s. in Static and Dynamic lithium.
- D P 14 The Effect of Irradiation at 550°C on the Tensile and Impact Properties of Austenitic and Martensitic Steels
- D P 15 The Effect of Irradiation at 150 and 250°C on the Tensile Properties of Cu and Cu CrZr
- D P 16 Magnetic Phase Induced by Plasma Disruption in AISI 316 SS
- D P 17 Stored Energy in MgO and Al₂O₃
- D P 18 Tensile and Fatigue Properties of 316 LN Austenitic Steel at Cryogenic Temperatures
- D P 21 Quantification of Plasma Induced Defects in First Wall Materials
- D P 22 Behavior of Graphites Under Intense Loads
- D P 23 Electron Beam Disruption Simulation of Refractory Metals
- D P 24 High Heat Flux Tests of Graphite for Limiter Applications
- D P 25 Metal/Graphite – Composite Materials for Fusion Devices
- D P 26 Siena : a Spectrum Tailoring Device for Fusion Materials Irradiation in a MTR
- D P 27 Neutron and Deuteron Irradiation Creep in Stainless Steel Alloys

- D P 40 TEXTOR Neutral Injector Beam Line Design
- D P 41 Feedback Control System for Stable Ion Beam Extraction
- D P 42 Achievements with the ASDEX Neutral Beam System During its Operation
- D P 43 Calculations and Measurements : Ion Removal System in the ASDEX/W VII – AS Long Pulse Neutral Injection Beam Line
- D P 44 Reflex Type Sheet Plasma Negative Ion Source
- D P 45 Test of a RF – Ion Source with a Large Scale Extraction System
- D P 46 Inertia Targets and Acceleration Grids for NBI – TEXTOR
- D P 47 Performance of the First JET Neutral Beam Injector
- D P 48 A Fast Shutter System for the JET Neutral Injectors
- D P 49 TORE SUPRA Injector Ion Source
- D P 50 Neutral Beam Injector for TORE SUPRA
- D P 51 The TARA Neutral Beam Fault Detection and Control System
- D P 52 Operational Test of the Second JET Neutral Injector with Deuterium Beams at Extraction Voltages of $< 160\text{kV}$
- D P 53 Interaction of Charged Particle with Fusion Material

POSTER PRESENTATION

Wednesday morning

Session E : **Experimental systems (Part II : oral)**
Blanket Technology – Refueling – Safety

- 08.30 h E P 01 The Compatibility of H₂O with Li₇Pb₂ and its Interactions with Hydrogen Isotopes
- E P 02 Optimal use of Beryllium for Fusion Reactor Blankets
- E P 03 Interaction of Hydrogen Isotopes with the Liquid Eutectic Alloy 17Li83Pb
- E P 04 An Alternative Neutronics Solution for Liquid Lithium Blankets
- E P 05 Beatrix – The International Breeder Materials Exchange
- E P 06 A Manufacturing Feasibility Study of Water Cooled Liquid Breeder Blanket Modules for NET
- E P 07 Compatibility Between Li – Ceramics and Beryllium
- E P 08 A Low – Risk Aqueous Lithium Salt Blanket for Engineering Test Reactors
- E P 09 Transient Analysis of the Water – Cooled Eutectic LiPb Blanket of NET
- E P 10 Thermal and Stress Analysis of a Solid B.O.T. Blanket for NET
- E P 11 Effects of Openings in a Fusion Reactor Blanket (Tokamak Type)
- E P 12 MHD Effects on Heat Transfer and Temperature Fluctuation of Liquid Lithium
- E P 13 Macrostructure, Microstructure and Electrical Properties of Porous – LiAlO₂
- E P 14 Neutronic Experiments in a LI – PB Spherical Assembly
- E P 15 Ceramic Tritium Breeder Materials – Promise and Performance
- E P 16 Thermal Properties and Application of Potential Lithium Silicate Breeder Materials
- E P 17 Progress in Fusion Reactors Blanket Analysis and Evaluation at CEA
- E P 18 Helium – Cooled Lithium – Lead DEMO Reactor Blanket
- E P 19 Liquid Metal Cooled Blanket Concept for NET
- E P 20 Progress in Design and Analysis of the NET Water Cooled Liquid Breeder Blanket
- E P 21 Design and Thermohydraulic Optimization of a Solid Ceramic Breeder Blanket for NET
- E P 22 Il Mantello – A Helium Cooled Solid Breeder Blanket for NET
- E P 23 First Wall Risk Analysis for a Loss of Coolant Accident in an Experimental Fusion Reactor
- E P 24 Activation Studies for Potential Structural Materials of Fusion Reactors
- E P 25 Tritium Atmospheric Release : A Simple Model for the Evaluation of HTO Formation
- E P 26 Safety Analysis Related to the Possible Release of Activated Erosion Dust in Fusion Reactor
- E P 27 First Wall and 17Li – 83Pb – Blanket Systems Accidental Analysis for NET
- E P 28 The Implications of Dose Rate Limits for the Recycling of Fusion Reactor First Wall Structural Material

- E P 40** Device for Varying Pellet Size in a Centrifuge Pellet Injector
- E P 41** D2 Pellet Injector for the Reversed Field Pinch ETA – BETA in Padova
- E P 42** On Arc Heated Gas Gun Acceleration of Pellets
- E P 43** Experimental Test of 6mm Diameter D2 Pellets Produced by in – situ Condensation
- E P 45** Pneumatic Pellet Injectors for TFTR and JET
- E P 46** Development of a Two – Stage Injector for Solid H₂ Pellets

POSTER PRESENTATION

Thursday morning

Session G : Tritium Handling – Data Acquisition and Control Systems

- 09.45 h G P 01 The COMPASS Data System
- G P 02 The COMPASS Control and Engineering Monitoring System
- G P 03 The AMOS/D Data Analysis System for ASDEX UPGRADE
- G P 04 Some Features of the Data – Handling System in Rijnhuizen
- G P 05 Electronics for Data Preparation and Diagnostics Control in ASDEX
- G P 06 JET Pulse Termination Network
- G P 07 Distributed Intelligence in a lan Architecture Increases the Flexibility in control Systems for Fusion Experiments
- G P 08 TORE SUPRA Control Computer System
- G P 09 Application of the Fast Array Processor for JT – 60 Plasma Control
- G P 10 Structure of the RFX Control and Monitoring System
- G P 11 Data Acquisition and Processing Systems at the NOVA High Energy Laser Facility
- G P 13 Commissioning and Operation of the Control, Safety and Interlock on the JET Neutral Beam Injector
- G P 14 The Diagnostic Subsystems of the AMOS/D System
- G P 15 Computer – to – Process Interface in JET Control and Diagnostic Systems
- G P 16 A Fast Signal Acquisition System for the Time Resolved Lyman – Alpha Diagnostic on TEXTOR
- G P 17 Development Status of FTU Control System
- G P 18 The TORE SUPRA Data Acquisition System
- G P 19 The TARA Control, Data Acquisition, and Analysis System
- G P 20 The JET Fast Transfer Diagnostic System – Installation and Commissioning

- G P 40 Tritium Release from Neutron Irradiated Li_7Pb_2 and $\text{Li}_{17}\text{Pb}_{83}$
- G P 41 Air Detritiation Systems for Fusion Facilities Based on CANDU Developments
- G P 42 Development of an Electrolysis Cell for the Recovery of Tritium from its Oxides
- G P 43 Impact of Bubble Nucleation on Tritium Permeation from Blanket Containment
- G P 45 Solidification of Tritiated Waste
- G P 46 Removal of Hydrogen from a Gas Stream by Noble Metals Embedded in Protective Supports
- G P 47 Out of Pile Tritium Release from Various Lithium Materials
- G P 48 An Assessment of the Tritium Inventory in, Permeation Through and Releases from the NET Plasma Facing Materials
- G P 49 Experimental Determination of the Kinetic Conversion Rate of Gaseous Tritium into HTO
- G P 51 A Ceramic Container for Hydrogen Purification on Hot Uranium
- G P 52 Tritium Release from Various Solid Breeding Materials
- G P 53 New Concepts for the Recovery and Isotopic Separation of Tritium in Fusion Reactors
- G P 54 Progress in Tritium Technology in the IPP T – Laboratory Scale – Up of Gas Chromatograph System ; Removal of Tritium from Secondary Containments
- G P 55 Tritium Separation from LiPb Blankets by Permeation into a Gas Stream or into NaK
- G P 56 Hydrogen Extraction from a Gas Mixture
- G P 57 Torus Evacuation and Tritium Handling on NET : Requirements, Design Approaches and Development Issues

POSTER PRESENTATION

Thursday afternoon

Session H : Magnet Technology – Reactor Studies – Remote Handling

- 15.35 h H P 01 Shear Strength of the ASDEX UPGRADE TF – Coil Insulation : Rupture, Fatigue and Creep Behaviour
- H P 02 Laser Beam Welding of Advanced Superconducting Cable Conduits
- H P 03 Geometrical Considerations for the Bending of Superconductors for Planar and Nonplanar Magnets
- H P 04 TFTR Control Systems for Protection and for Normal Control of Magnetic Field Coils
- H P 05 Mechanical Behaviour of Medium Sized Modular Stellarator Coil Configurations
- H P 06 An Improvement of the Flange Bolting in the FEM – Model of the EURATOM – LCT Coil and Comparison with Results Measured in the TOSKA Test
- H P 07 The JET Magnets : Operational Experience and Plans for Upgrade
- H P 08 Different Manufacturing Processes of Internal Connections of Toroidal Field Coils
- H P 09 The Central Solenoid of NET
- H P 10 Manufacturing of the LN₂ Cooled Toroidal Coils of FTU
- H P 11 A New Concept in the Design of HighField Toroidal Magnets
- H P 12 Fabrication and Test Results of the Supraconductor for the Toroidal Field Coils of TORE SUPRA
- H P 13 Tests of the Toroidal Field Coils of TORE SUPRA
- H P 14 Manufacture and Transport of the TORE SUPRA Large Poloidal Fields Coils
- H P 15 Manufacture and Test of the TORE SUPRA OHMIC Heating Coil
- H P 16 Development of a Forced – Cooled Superconducting Coil
- H P 17 Design, Construction and Test on a Prototype of a Poloidal Coils of the FTU
- H P 18 Dimensional Accuracies Obtained in the TORE SUPRA Toroidal Field Coils
- H P 19 Contact Resistance of Demountable Multi – Pin Joint for Superconducting Helical Coil
- H P 20 Response of Superconducting Coils to Internal Heating
- H P 21 Testing Electrical Insulation of LCT Coils and Instrumentation
- H P 22 The TARA 24 MVA Magnet System
- H P 23 First Results of the TESPE – S Magnet System Safety Experiments
- H P 24 ASDEX UPGRADE Poloidal Fields Coils – Concept, Manufacturing, Test
- H P 25 Design of Superconducting Pulsed Coil by a CAD System
- H P 26 COMPASS Magnetic Field Coils and Structure Systems
- H P 27 A Low Loss Superconducting Cable for Poloidal Field Coils and its Application in a 3m⁰ Model Coil
- H P 28 Progress in the NET Toroidal Field Coil System Design
- H P 29 Experience with Cooldown of Superconducting Magnet Coils in the Large Coil Task
- H P 30 Considerations on Magnet Design Based on Forced Flow of Helium II in Internally Cooled Cables
- H P 31 Stress Analysis of Conductor and Winding Package at the Central Vault of the SIN Toroidal Field Coil for NET

- H P 40** The In – Vessel Inspection System on the JT – 60
- H P 41** The JET In – Vessel Inspection System
- H P 42** Remote Handling Tools for JET
- H P 43** Tokamak Reactor Configuration and Maintenance
- H P 44** Plant Design for Reversed Field Pinch Reactor (TITAN)
- H P 45** On Some Interesting Properties of the Working Temperature in a Tokamak Reactor
- H P 46** Studies on Ignition Target and Reactor in ICF Research
- H P 47** The Promise of Tritium Lean Fuels
- H P 48** The Dense Z Pinch as a Fusion Power Reactor : Breeding and Radiation Damage Considerations
- H P 49** Light Ion ICF Reaction Chamber Mechanical Design
- H P 50** TIBER II Configuration and Structural Design